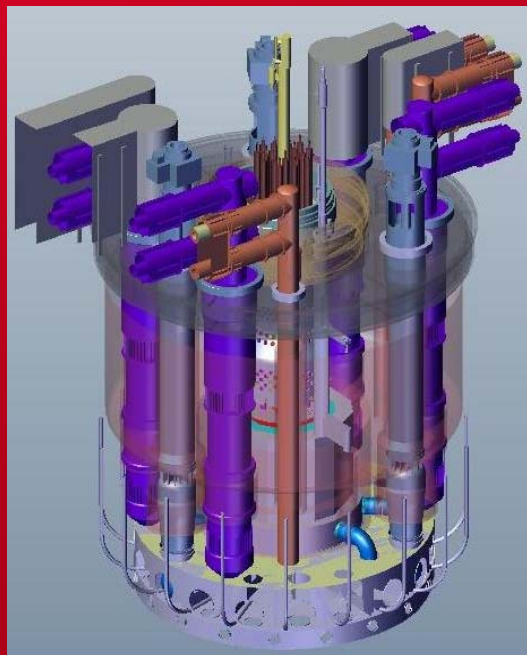


DE LA RECHERCHE À L'INDUSTRIE



[www.cea.fr](http://www.cea.fr)

## ADVANCED SODIUM TECHNOLOGICAL REACTOR FOR INDUSTRIAL DEMONSTRATION

### THE ASTRID PROJECT

#### STATUS, COLLABORATIONS, LESSONS FROM FUKUSHIMA ACCIDENT

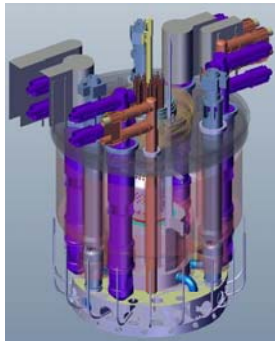
International Workshop on Prevention and Mitigation of Severe  
Accidents in Sodium-cooled Fast Reactors

11th-13th June, 2012, Tsuruga, Japan

François Gauché

JUNE 12<sup>TH</sup>, 2012

- Excellent use of uranium resource and ability to recycle plutonium without limitation (multirecycling).
  - Unlike the vast majority of reactors currently in operation or in construction worldwide, using only about 1% of natural uranium, fast neutron reactors are able to use more than 80% of the uranium resource. For instance, the current stockpile of depleted uranium available on French territory could feed the needs for electricity production at current rate for thousands of years.
- Ability to perform transmutation and burning of certain minor actinide
- Safety objectives equivalent to other reactors that are put into service at the same time
  - For ASTRID prototype, this means a safety level at least equivalent to 3rd generation reactors associated with the integration from design on of lessons learnt from Fukushima accident;
- Objective of good economic competitiveness for commercial use
  - Depending on the service that is rendered
- Objective of good guarantees in terms of proliferation resistance

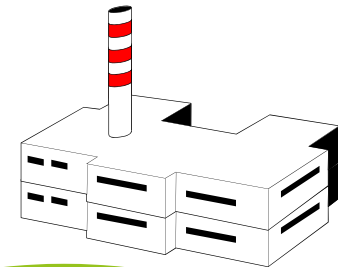


## ASTRID design studies

- Industrial demo/prototype 600 Mwe
- 4<sup>th</sup> generation reactor
- Irradiation tool

## Core fabrication workshop

- MOX fuel
- A few tons per year

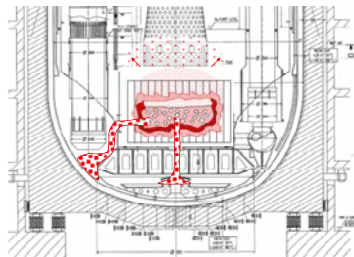


## Full scale component testing

- Refurbishment/Realization of large test sodium loops
- Refurbishment of zero power reactor MASURCA



## Severe accidents experimental program

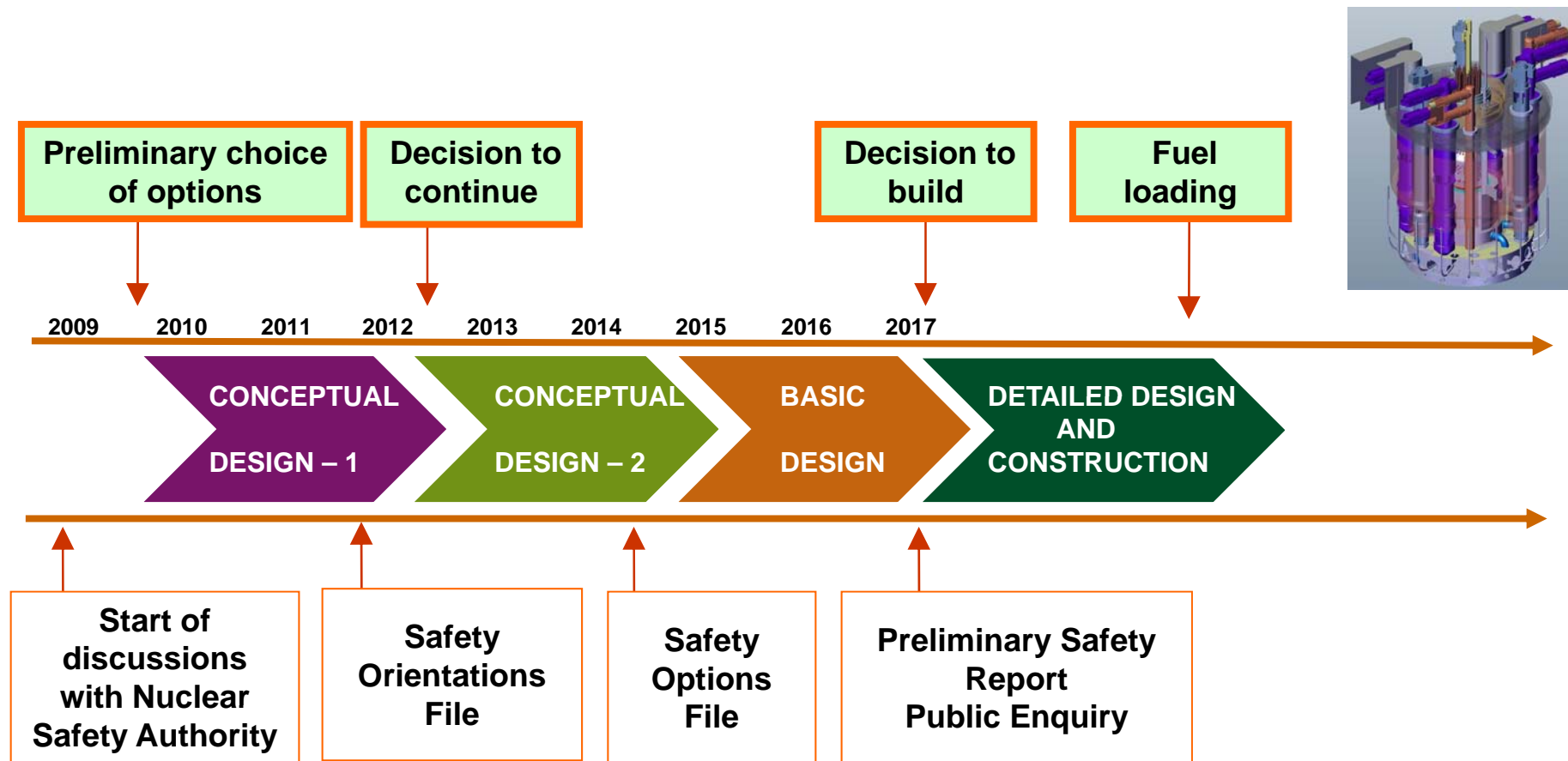


**+ R&D**

(including fuel cycle)

Feasibility study for minor actinides  
bearing experimental fuel fabrication

# SCHEDULE FOR ASTRID AND LICENCING



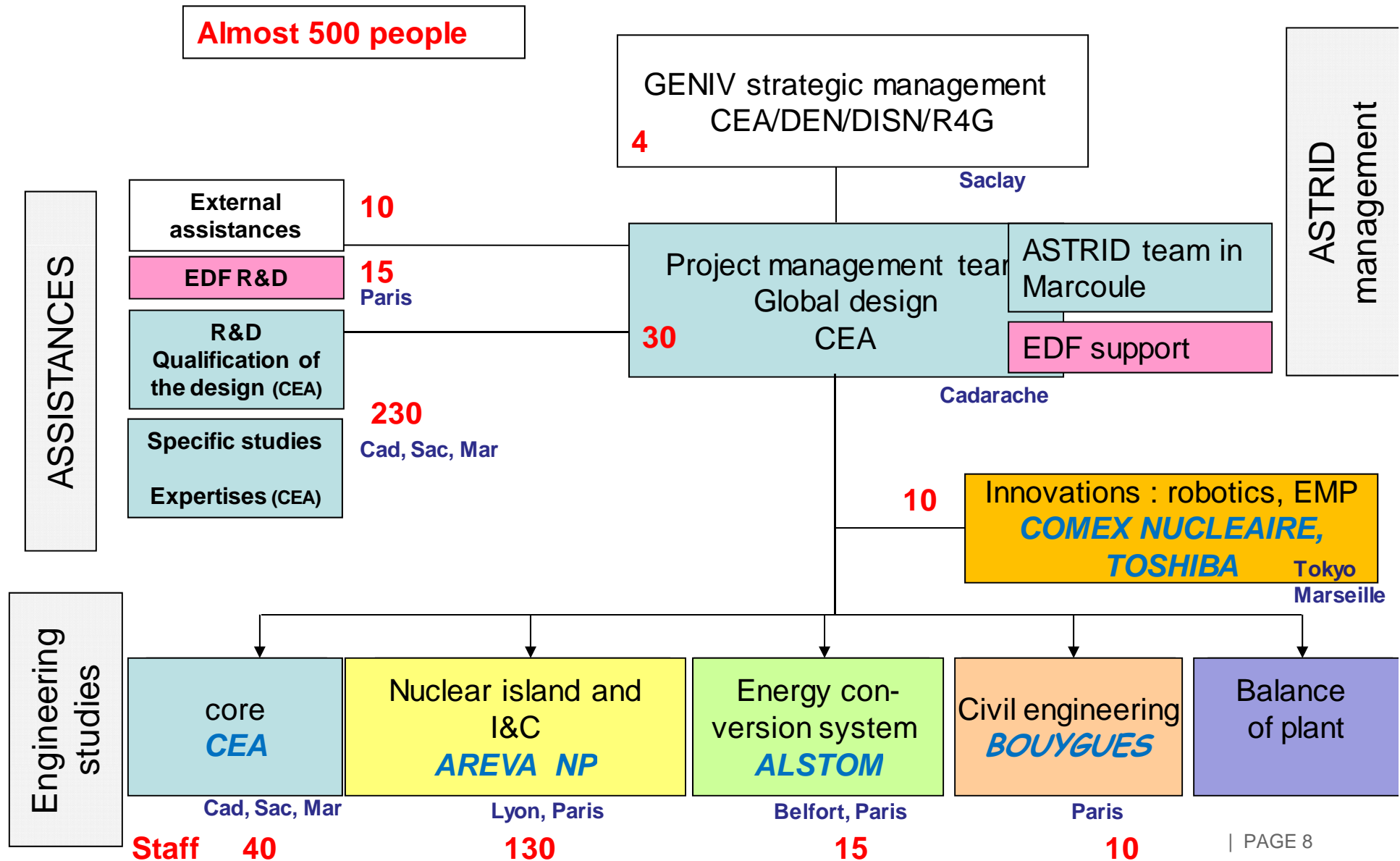
- Conceptual design Phase 1 started in 2010
- 1st important deadline: End of 2012. Studies are on schedule.
- Several partnerships signed: AREVA, EDF, ALSTOM, COMEX Nucléaire, TOSHIBA, BOUYGUES... Other discussions going on.
- More than 500 people working on the project (R&D and design)
- Safety Orientations Document to be submitted to French Nuclear Safety Authority by the end of June 2012.
- Check of Coherence of Safety Orientations with draft GIF Safety Design Criteria to be completed by the end of June 2012.
- Promising design options: core, energy conversion system, confinement, in-service inspection.
- Preparation of launch of Conceptual Design Phase 2 (2013-2014)
- Update of R&D program to meet ASTRID needs. Expression of interest toward international collaborations on specific items.

- ASTRID will take into account the lessons learnt from Fukushima accident from the design on.
- The design will build upon the advantages of pool-type, sodium-cooled fast neutron reactors, i.e. favourable intrinsic features to cool the reactor: large thermal inertia, diversified heat sink, natural circulation, ability to guarantee minimum sodium level.
- Safety objectives of ASTRID are derived from WENRA\* statement “Safety objectives for new nuclear power plants”. This document summarizes the highest standards, even in the case of Fukushima-like initiators. Former BDB or DEC events are to be considered in the design.
- As GENIV reactor, ASTRID will provide for significant improvement with regard to former SFR designs.

\*Western European Nuclear Regulator's Association

- The primary system is not pressurized.
- Power control by single rod position, no xenon effect, no need of soluble neutron poison.
- The intermediate (or secondary) system provides for an extra containment between the primary circuit and the environment.
- Large boiling margin of sodium (>300K).
- The large quantity of primary coolant provides for a high thermal inertia in case of loss of main heat sink.
- Good natural convection and circulation features allow to design passive, diversified decay heat removal systems.
- Collective dose on a pool type SFR is very low compared to PWR.

# ASTRID PROJECT INDUSTRIAL ORGANIZATION





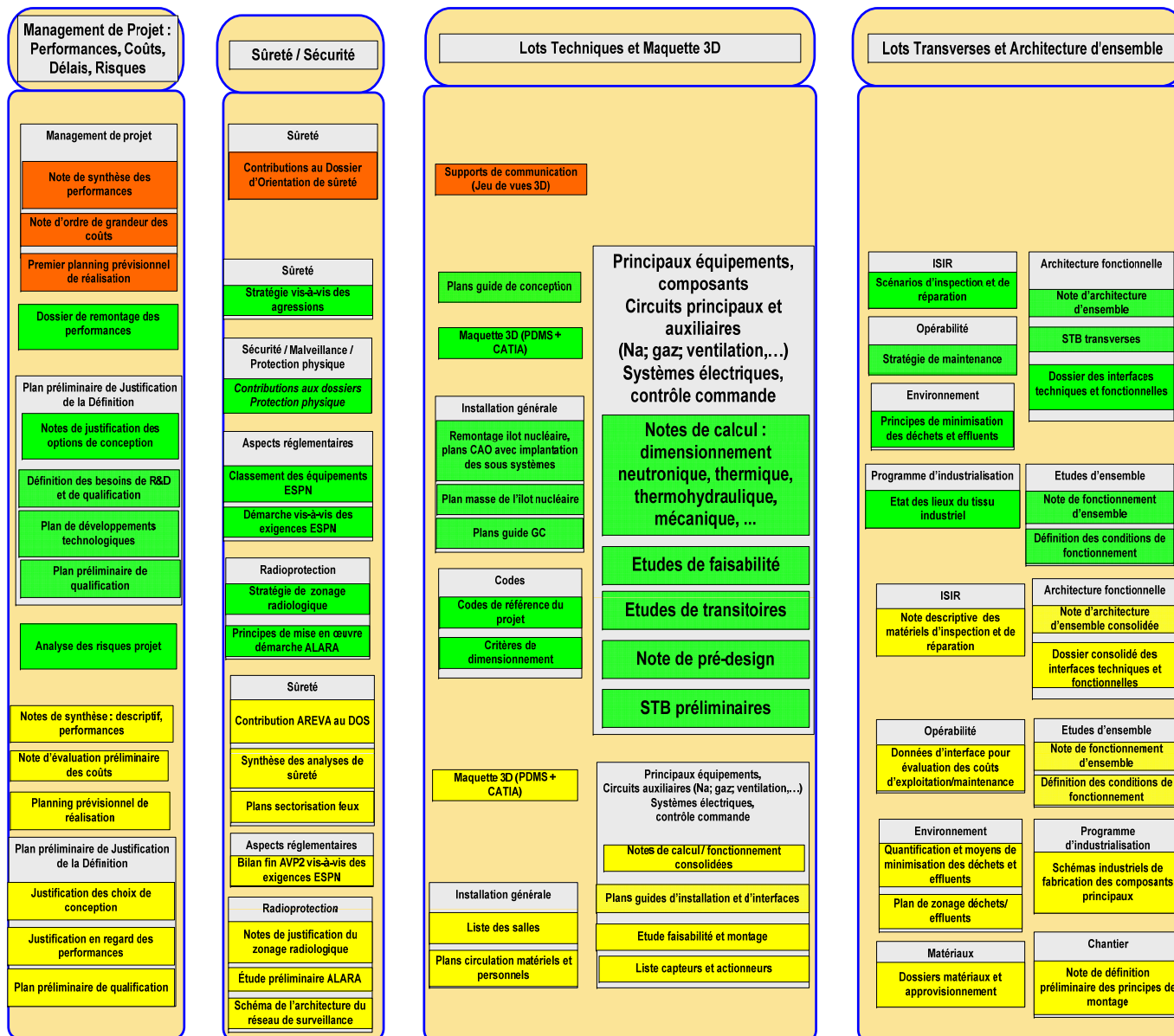
# CONTENT OF THE 2012 & 2014 FILES

Government  
2012 report

Pre-conceptual design

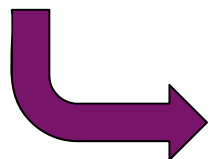
AREVA  
exemple

Conceptual  
design



# CONCEPTUAL DESIGN PHASE 1: PREDICTIVE NUMBER OF DOCUMENTS

	<i>management</i>	<i>safety</i>	<i>Technical reports</i>	<i>Interfaces &amp; general layout</i>	<i>Total</i>
<b>AREVA</b>	20	87	627	92	826
<b>CEA core</b>	11	20	102	6	139
<b>ALSTOM</b>	5	0	57	8	70
<b>COMEX</b>	5		10	1	16
<b>Civil works</b>	7	0	53	10	70
<b>Total</b>	48	107	849	117	<b>1121</b>



More than 1,000 technical documents up to 2012  
About 50,000 pages...

Feedback of previous SFR	R&D directions	ASTRID Orientations
<b>Core reactivity</b> <b>Issue of sodium void coefficient</b> <b>→ Safety</b>	<p>Optimization of core design to improve natural behavior in case of abnormal transients.</p> <p>Exploration of heterogeneous cores</p>	<p>CFV core (patented in 2010): innovative approach, negative overall <u>sodium void coefficient</u>.</p> <p>Better natural behavior of the core, for instance in case of loss of cooling (e.g. due to loss of supply power)</p>
<b>Sodium-Water reaction</b> <b>→ Safety - Availability</b>	<ul style="list-style-type: none"> <li>- modular Steam Generators</li> <li>- Reverse Steam Generators (sodium in tubes)</li> <li>- Gas Energy Conversion System (nitrogen in place of steam/water)</li> </ul>	<ul style="list-style-type: none"> <li>- Limitation of total released energy in case of sodium-water reaction</li> <li>- Limitation of wastage propagation</li> <li>- Design studies conducted by ALSTOM. No show stopper.</li> </ul>

# STRONG IMPROVEMENTS IN SFR DESIGN FOR ASTRID 2/2

Feedback of previous SFR	R&D directions	ASTRID Orientations
Sodium fire → Safety	Innovation on sodium leak detection systems R&D on sodium aerosols	Improvement of detection (patent of detection system integrated of heat insulation) Close containment (limitation of available oxygen, inert gas)
Severe accidents → Safety	-Core catcher -R&D on corium and sodium-corium interaction	Core catcher. Several locations are under study (in vessel, ex-vessel or between the two vessels).
Decay heat removal → Safety	-Reactor vessel auxiliary cooling system (extrapolability)	Combination of proofed DHR systems, RVACS
In-Service Inspection and Repair → Safety – Availability	-Simplification of primary system design -ISI&R is taken into account from design on -New techniques : acoustic detection, LIBS, CRDS -Signal processing -TUSHT (ultrasound, high temperature), High temperature fission chambers, Optical Fibers, Flow meters for subassemblies -Remote handling for inspection or repair -Under sodium-viewing	

## ■ Russia

- Material irradiations in BOR-60, Neutronics in BFS
- Possibility of fuel irradiations in BN-600 ?

## ■ India

- Strong collaboration, several Implementing Agreement on going on safety, including severe accidents

## ■ USA

- Several STCs
- Safety benchmark on CFV core

## ■ Japan

- Several STCs
- EAGLE collaboration

## ■ Europe

- CP-ESFR project, SOFIA proposal
- Next Framework program in preparation (2014-2020)

## ■ Korea, China

- The 1st phase of ASTRID conceptual design is almost complete. Important results have been achieved, so that ASTRID shows major improvements with regard to past SFR.
- The industrial collaboration around ASTRID is of outmost importance, provides innovation, and ensure industrial stakes are taken into account.
- Design studies are on schedule.
- It is essential to launch 2nd phase of conceptual design, to keep the momentum that has been gathered on R&D and design activities inside CEA and among its industrial partners.
- On the side of technical developments and experimental facilities, CEA is willing to develop international partnerships

